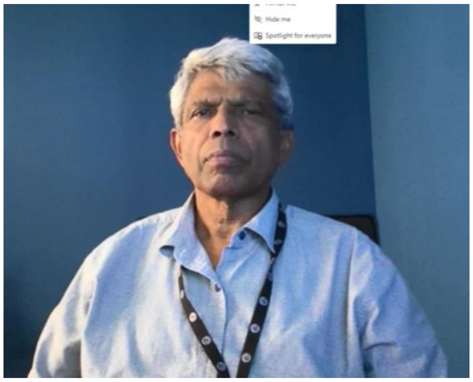
Dr. Hideo NAKAMURA, the chair of WGAMA, is technical associate of the Nuclear Safety Research and Emergency Preparedness Institute in Japan Atomic Energy Agency (JAEA). Graduated from the Nagoya University, Department of Nuclear Engineering Crystalline Materials Science (Fusion Materials) in 1981, he joined the ROSA (Rig of Safety Assessment) program at former JAERI to study reactor T/H during BDBAs of both BWR and PWR under reactor prototypical conditions. He became a head of the T/H Safety Research Group including SA in 2001, started the NEA/ROSA and ROSA-2 Projects with LSTF experiments in 2005, and has supported the Fukushima-Daiichi Accident tasks since 2011. Meanwhile, he has continued fundamental T/H safety research to assure safety assessment code reliability, including gas-liquid two-phase flows in large-diameter horizontal and vertical pipes, and the WGAMA scaling SOAR, core exit temperature (CET) effectiveness report and specialist SAMMI-2020 WS on the AM measurement methods. He was an executive editor of Nuclear Engineering and Technology (NET, 2015-2021). Currently, he is a deputy director of the Nuclear Safety Division of the Atomic Energy Society of Japan (AESJ), a member of the AESJ Ethics Committee and an AESJ Fellow.

Dr. Vinh N. DANG, the chair of WGRISK, is Head of the Risk and Human Reliability Group at the Paul Scherrer Institute (PSI). With M.S. und Ph.D. degrees in Nuclear Engineering from the Massachusetts Institute of Technology (MIT), he joined the PSI in 1994 where he has led research and regulatory support activities related to Human Reliability Analysis (HRA) and Probabilistic Safety Assessment (PSA). His research interests include the treatment of decision performance in Human Reliability Analysis, the collection and application of data from simulator (training) facilities, and the development of probabilistic scenario models for safety and resilience assessment for energy infrastructure. He served on the Steering Committee and the Assessment Group in the International HRA Empirical Study carried out at the OECD Halden Reactor Project (2006-2012). He served as the Swiss delegated representative to OECD Halden for Man-Technology-Organization topics (Halden Programme Group) from 2006 to 2018. He currently lectures at the Swiss Federal Institute of Technology, Zurich (ETHZ), on Probabilistic Safety Analysis and Human Reliability Analysis.



Dr. Thambiayah (Nithy) NITHEANANDAN earned his Masters and Doctoral degrees in Mechanical Engineering from the University of Manitoba. Subsequently, he joined Atomic Energy of Canada Limited (AECL), a federal crown corporation, in 1993 and worked for 25 years. During his employment at AECL, he managed the Nuclear Safety Experiments branch, which consisted of a team of engineers, scientists, and technologists. He led medium- and large-scale experimental projects such as the Molten Fuel Moderator Interaction project and the Qinshan contact boiling experimental program. He chaired the Nuclear Energy Agency's Working Group on Analysis and Management of Accidents for six years.

In 2018, he became a director of the Engineering Design Assessment Division of the Canadian Nuclear Safety Commission, Ottawa. Currently he is the Director of Reactor Behaviour Division. He is an industry professor in the Department of Engineering Physics at McMaster University, Hamilton, Ontario.

His expertise includes steam explosion studies, pool boiling, high-temperature heat transfer, severe accidents, supercritical water reactor design, reactor safety, and thermalhydraulics.

Dr. Ahmed BENTAIB has been a nuclear safety expert at IRSN since 2000. He earned his Ph.D. from Lyon University in 1993 and obtained the Accreditation to Supervise Research (HDR) from the University of Orléans in 2020.

His research focuses on severe accidents, hydrogen dispersion modelling, as well as mitigation and combustion phenomena relevant to the safety of nuclear and industrial facilities. He has published over 100 papers in peer-reviewed journals and international conferences and has contributed to the preparation of three books.

Throughout his career, Dr. BENTAIB has participated in numerous national, European, OECD, and IAEA projects. With more than 20 years of professional experience, he currently serves as the Vice-Chair of the NEA Working Group on the Analysis and Management of Accidents (WGAMA), co-leader of NUGENIA’s Technical Area 2 on Severe Accidents, and Chair of the ETSON expert group on severe accidents.

In addition to his research, Dr. BENTAIB has collaborated with various universities and currently teaches master’s courses on nuclear safety. He has supervised Ph.D. students, post-doctoral researchers, and several master’s students.

Prof. Francesco D'AURIA had born 1954, received Master degree in Nuclear Engineering in 1978 and PHD degree in Nuclear Reactor Safety in 1980 both at University of Pisa. He was awarded of a research grant from University of Pisa in 1978. He was winner of the Assistant Professor position in 1981 at University of Pisa (area of nuclear thermal-hydraulics), of the Associate Professor position at University 'La Sapienza' in Rome in 1991(area of radiation applications) and the Chair (Full) Professor position at University of Pisa in 2000 (area of nuclear thermal-hydraulics), the last one being the current position. He was engaged as 'Adjoin Professor' at University of Genova in the period 1993-2003 (area of world energy status and planning). He is ANS member since 1981. Since 1980 he has been member in several international working groups at ANS, EC, IAEA and OECD/NEA including CSNI since 2015 and CSNI Senior Group of experts since 2017. He was member and Chair of Technical Program in several International Conferences and member of the scientific committee for a dozen Journals. He is recipient of various recognitions, e.g. best paper, excellent session organizer, most downloaded paper notification, Marquis Who's Who VIP listee (selected for Albert Nelson Marquis Lifetime Achievement Award) etc. Key awards and achievements:

- Foreign Member of Argentinean Academy of Science in Buenos Aires (Argentina), since 2008.
- Recipient of "N.A. Dollezhal Gold Medal" by NIKIET in Moscow (Russia), 2006.
- Founder and Editor in Chief (during 5 years, since 2005) of the Hindawi Journal Science and Technology of Nuclear Installations (J STNI).
- Editor-in-Chief of Elsevier Journal Nuclear Engineering and Design (J NED) since 2023; Editor since 2019; Editorial Board Member since 1990.
- Member of Editorial Board of a dozen Scientific Journals including ANS J Nuclear Technology and ANS J Nuclear Science and Engineering, since 2024.
- Recipient of Technical Achievement Award (TAA) by American Nuclear Society in 2021 (motivation: Enhancing and promoting the best use of computational tools in nuclear reactor safety and licensing: methods for validation, uncertainty, scaling and design of experiments, reliability of passive systems. Integrating fundamental technology and understanding with applications).
- ANS Fellow nomination award in 2024.
- General Chair of ANS NURETH-15 Conf., Pisa (Italy) 2013, NURETH-21 (Co-Chair) Conf., Busan (Korea) 2024, ENS TopSafe Conf.s. Dubrovnik (Croatia) 2008, Helsinki (Finland) 2012, and Vienna (Austria) 2017, OECD/NEA/CSNI Spec. Meet., Madrid (Spain), 2022.
- Editor and key Author of the Elsevier Book "Thermal Hydraulics of Water Cooled Nuclear Reactors", 2017. Co-Editor and key Author of the three Volumes Elsevier "Handbook on Thermal Hydraulics of Water Cooled Nuclear Reactors", 2024.

- Recipient of ANS NURETH Scholar award in Xi'an (PRC), 2017.
- Coordinator-promoter-author of four State of Art Reports (SOAR) published by OECD/NEA: Two-Phase Critical Flow (TPCF) in 1980, Thermal-hydraulics of Emergency Core Cooling Systems (TECC) in 1987, Boiling Water Reactor Stability (BWRS) in 1996, Scaling (S-SOAR) in 2017 and Reliability of Passive Systems (2024).
- Member of NEA scientific groups since 1980 (PWG-2, W-GAMA, etc.); CSNI member since 2016, until 2024; member of CSNI Senior Expert Group 2017-2020.
- Member of IAEA scientific and consultancy groups period 1983-2016.
- Life recognition for activities in BEPU area, (Lucca, BEPU2018).

Co-author of 1200+ Papers (IRIS ARPI): 250+ Journal papers; 600+ Conferences; 100+ IAEA and OECD International Reports; 10+ Books.

Dr. Fulvio MASCARI has a Master's Degree in Nuclear Engineering (2006) and the doctorate PhD in "Nuclear, Chemical and Safety Technology" (2010) at the University of Palermo.

Since September 2013 he has worked as a researcher at the ENEA. His technical field is the nuclear reactor thermal-hydraulics in reactor coolant systems/containment, and their coupling and the analyses of severe accident phenomena. In relation to that, he is an expert on the use of best estimate thermal-hydraulic system code (RELAP5 and TRACE) and severe accident code (ASTEC and MELCOR).

Currently he is investigating severe accident issues in PWR and BWR reactor types, the analyses of the capability of best estimate thermal-hydraulic system code to simulate the main phenomena typical of advanced light water reactor, as small modular reactor, and the thermal-hydraulic phenomena typical of fusion reactor, the analyses of the scaling issues, the analyses of the capability of severe accident code to simulate degradation phenomena, and the application of the probabilistic method to propagate input uncertainty in deterministic safety analyses.

He is involved in several international activities (e.g. EURATOM Projects, NEA/CSNI/WGAMA activities, IAEA activities, etc). He is currently the coordinator of the Horizon Euratom SASPAM-SA project.

Dr. Joy REMPE is Principal, Rempe and Associates, LLC. She has over 35 years of experience in reactor safety and instrumentation performance. Prior to retiring as a Laboratory Fellow at Idaho National Laboratory (INL), she founded an instrumentation development and deployment laboratory for U.S. and international irradiation test facilities. During her INL tenure, she also led numerous severe accident research projects, including several Three Mile Island Unit 2 post-accident activities. As Principal of Rempe and Associates, LLC, she provides consulting to U.S. and international organizations. Since 2014, she serves as technical lead for DOE Fukushima Daiichi forensics efforts.

Authoring over 400 publications/presentations, she holds 4 patents (one winning an R&D 100 award). Dr. Rempe is an American Nuclear Society (ANS) Fellow. Between 2010-2023, she served on the NRC Advisory Committee on Reactor Safeguards (ultimately, Chairman). Between 2013-2019, she served on the DOE Nuclear Energy Advisory Committee (ultimately, Co-Chair). She is the NEA Fukushima Daiichi Nuclear Power Station Accident Information Collection and Evaluation (FACE) Project Program Review Group Chair. She holds nuclear engineering degrees from Massachusetts Institute of Technology (MS and PhD) and University of Missouri – Rolla (BS).

Dr. Pierre RUYER works in the field of thermal hydraulics for nuclear safety issues since 2006 and is a research director at the Autorité de Sûreté Nucléaire et de Radioprotection, ASNR, France. Specialist of two-phase heat and mass transfer, he tutored a dozen of PhD and co-authored approx. 25 journal papers. The scientific topics of its interest are film boiling, explosive vaporization, droplets flow, radiative heat transfer, natural circulation, evaporation, nucleation, bubbles and droplets hydrodynamics and size distribution, granular flows. He studied them thanks to a mix between experimental and numerical methods. He provided its expertise in several assessment process of safety studies. Safety issues related to his work concerns LOCA, RIA, Spent Fuel Pools, Passive Systems, BORAX. He participated to several European projects and to the steering committee of the French NEPTUNE project. He has been involved in several CSNI activities within WGFS and WGAMA. Since 2018, he coordinates the task group on CFD within WGAMA and since 2021 is member of the bureau of WGAMA.
Dr Pierre RUYER holds a PhD degree in Mechanics from Ecole Polytechnique from 2006 and a French habilitation degree in Science at Aix-Marseille University from 2022.

Dr. Didier JACQUEMAIN is a Senior nuclear safety specialist at the NEA since January 2020. He is a technical advisor for the working groups active under the Committee on the Safety of Nuclear Installations (CSNI) and a technical co-ordinator for the NEA joint nuclear safety research projects. Before joining NEA, he worked for 27 years at the French Institut de Radioprotection et de Sûreté Nucléaire (IRSN) in the nuclear safety research area and was mostly involved in research programmes on fuel safety, severe accidents and ageing with an active contribution to international research projects in nuclear safety. He received a PhD in physical chemistry from the Weizmann Institute of Science (Israel) in 1992 and graduated from the École Supérieure de Chimie Industrielle de Lyon in 1988. His book on Nuclear Power Reactor Core Melt Accidents was published in 2015.

Dr. Luis Enrique HERRANZ has a Ph.D. from the Polytechnique University of Madrid on Nuclear Safety (UPM) in 1996, Master in Nuclear Engineering in 1987 from the Institute of Energy Studies, and Graduate in Quantum Chemistry in 1986.

Research Professor since 2015, he has been leading the Unit for Nuclear Safety Research for 25 years. As a result of his research in national and international projects, he has published hundreds of papers in refereed journals in the areas of Severe Accidents, Nuclear Fuel Thermo-mechanics, Thermal-hydraulics of Passive Safety Systems and Power Cycles in Nuclear Technology. He has been the technical coordinator of some recent EURATOM projects in the area of nuclear safety.

He is currently the leader of the Technical Area 2 (Severe Accidents) of SNETP/NUGENIA and was the Chair of the WGAMA (Working Group of Analysis and Management of Accidents) under the NEA between 2016 and 2021. At present, he is member of: the Advisory Board of WGAMA, the Working Group on Fuel Safety (WGFS), and the Expert Group of Small Modular Reactors (EGSMR).

With a strong committed to teaching and training, he lectures regularly in national and international graduate and post-graduate courses.

Dr. Chul-Hwa SONG (C.-H. Song) is currently Distinguished Researcher working for KAERI over the four decades. Previous positions include VP for nuclear safety research department and the Director of TH safety research division. He got involved in numerous R&D programs on ALWRs, focusing on advancements of nuclear TH and reactor safety. He also led, as PI, the U.S.-ROK I-NERI joint program and the Korea-Hungary Joint Research Laboratory program. He is also professor at UST since 2008 and was chief major professor leading the major of nuclear system engineering for 10 years. He was adjunct research professor at UIUC, USA (2017-2018) and a visiting researcher in CEA-Grenoble, France (1987-1989).

He worked, as bureau member and representative of Korea, for CSNI and WGAMA, in which he led the IBE-2 CFD exercise as PI and supervised the OECD-ATLAS joint projects as director at KAERI. He played pivotal role for several programs, such as IBE-2 benchmark, CFD4NRS-4 workshop and 3-D SysTH program among others, as (co-)initiator or leader.

He is currently associate editor of JANDT since 2018 and was editors of NED (2014-2022) and of NET (2008-2015). He is also on int'l advisory board of JNST and KernTechnik journals. He served as chair of ANS THD and of diverse committees in int'l conferences. He is currently General Chair of NURETH-21 (2025). He is the TAA recipients of both ANS and KNS. He got the honor of NURETH Fellow and ANS Fellow, and the Order of S&T Merit from Korean government. He published 110+ papers in SCI journals and 60+ registered patents. He holds BS/MS degrees in ME from Hanyang univ. and Ph.D. in NE from KAIST, all in Korea.

Dr. Dominique BESTION has been Research Director at CEA-Grenoble, in France. Has worked extensively in modelling two-phase flow for system and CFD codes, and has been project manager of the CATHARE code development. He coordinated two-phase flow modelling in the NEPTUNE French multi-scale thermalhydraulic platform and he was coordinator of Thermalhydraulic activities of the NURESIM, NURISP and NURESAFE European Projects for a multi-disciplinary and multi-scale simulation platform. As a member of the OECD-CSNI- WGAMA, he coordinated a Task Group on the application of CFD to nuclear reactor safety issues. He also had teaching activities in France in several Universities, at the Ecole Polytechnique, at the INSTN, and in many international courses, including at ETH-Zurich and in IAEA courses in several countries. He was Editor at Nuclear Engineering and Design. He is now Scientific Advisor at CEA-Saclay and chairman of the FONESYS international network of system code developers.

Dr. Jean BACCOU is a senior expert on uncertainty analysis at the French Autorité de Sûreté Nucléaire et de Radioprotection (ASNR). After a Phd in Applied Mathematics from Aix-Marseille University, he joined the French Institut de Radioprotection et de Sûreté Nucléaire (IRSN, now ASNR) where he has been a research engineer for the past 20 years. His main field of interest is data science to handle nuclear safety problems. In particular, he has developed with his co-workers several methodologies to take into account uncertainties in nuclear systems and to predict complex spatial phenomena using machine learning techniques. He has been involved in several international projects on uncertainties and was the coordinator of the OECD SAPIUM project on input uncertainty quantification and validation.

Dr. Jinzhao ZHANG is a Global Expert and Technical Director at TRACTEBEL (ENGIE), where he leads activities in nuclear fuel design, safety analysis, and licensing. He holds a PhD in Mechanical Engineering from UCLouvain and brings over 40 years of experience in R&D, engineering, and consulting. Dr. Zhang is renowned internationally for his expertise in nuclear reactor thermal hydraulics, fuel design, and safety analysis.

Since the 1990s, Dr. Zhang has been an active member in the OECD/NEA Working Group on Analysis and Management of Accidents (WGAMA), the Working Group on Fuel Safety (WGFS), the Expert Groups on Thermal Hydraulics and Mechanics (EGTHM) and on Multi-physics (EGMUP). He also chairs the NEA Expert Group on Reactor Fuel Performance (EGRFP) and the IAEA Technical Working Group on Fuel Performance and Technology (TWGFPT). Dr. Zhang has spearheaded numerous international projects on thermal hydraulics, fuel performance and safety, including NEA's SAPIUM, DEC-A, RBHT, RIA benchmark, and SMR Fuel, as well as IAEA's ATF-TS CRP.

Throughout his long career, Dr. Zhang has published over 100 papers on topics such as two-phase flow and heat transfer, thermal hydraulics, multi-physics, code validation, uncertainty quantification, reactor core and fuel design, and safety analysis. He has also contributed to the recently published book, "Thermal-Hydraulic Principles and Safety Analysis Guidelines of PWRs and iPWR-SMRs."

Dr. Thorsten HOLLANDS is Chief Engineer in the field of reactor safety research and leads the Code Validation team at Gesellschaft fuer Anlagen- und Reaktorsichehrait (GRS). He has worked now for nearly 20 years in the areas of validation and development of ATHLET and ATHLET-CD at GRS and Ruhr-University Bochum, Germany. His main responsibility in the development covered the modelling of nitride formation during air ingress which is a unique feature of ATHLET-CD compared to other severe accident codes. In the last 14.5 years, the focus of his work has been on validation of GRS system codes simulating the primary circuit as well as application of AC^2. Since 2013, Thorsten is the project lead of the ATHLET and ATHLET-CD validation. He is the GRS project lead of the EC funded project TANDEM dealing with integration of SMR as co-generation plants in a grid including safety analyses. Furthermore, Thorsten is strongly involved in the EC funded project SASPAM-SA which deals with severe accident analyses for SMR. Additionally, he was involved in several EU projects about Gen-IV assessment of ATHLET, severe accident application of ATHLET-CD as well as in OECD/NEA (e.g. ATLAS-2/-3, ETAHRINUS, QUENCH-ATF) and IAEA projects and task groups concerning spent fuel pool behavior (e.g. Sandia Fuel Project, SFP-PIRT) and ATF. Since 2019 Thorsten is member of the OECD/NEA WGAMA and since 2022 member of the WGAMA Bureau.

Dr. Stephen M. Bajorek is the Senior Technical Advisor for Thermal-Hydraulics in the NRC's Office of Nuclear Regulatory Research and has nearly forty years of experience in the nuclear industry. He provides guidance for development of the TRACE state-of-the-art thermal-hydraulics code, advanced reactor analysis, and the NRC’s thermal-hydraulic test programs. He is currently leading the NRC's efforts to model and simulate advanced non-light water reactors. Dr. Bajorek represents the NRC in international thermal-hydraulic research projects and has served on U.S. State Department missions in support of U.S. industry interests.Prior to joining the NRC he was a member of the faculty at Kansas State University and has over 15 years of industrial experience as a code developer and analyst. He has authored or co-authored over 200 publications in areas ranging from boiling and two-phase flow, reactor safety, advanced nuclear plant analysis, natural convection, and boiling of multi-component fluids.

Dr. Bajorek received his Ph. D. from Michigan State University, with M.S. and B.S. degrees in Mechanical Engineering from the University of Notre Dame.

Dr. Andrew MORREALE is the Technical Manager – Reactor Safety, at Canadian Nuclear Laboratories (CNL), and previously spent 10 years as a research scientist focused on nuclear reactor severe accident modeling and analysis; having joined CNL in 2013. He is involved in many international efforts for OECD-NEA including chairing the CSNI Expert Group on Small Modular Reactors, participating in several projects focused on Fukushima Daichi Accident Analysis (BSAF-2, PreADES, ARC-F, FACE) and leading the FACE Round Robin Debris Analysis Activity. Andrew is also involved in IAEA NHSI Industry Track efforts focused on collection of experimental facility information in the to support SMR vendors in code validation and is the Steering Committee Chair for the associated NEXSHARE database. Dr. Morreale was the Technical Program Committee Chair of the NUTHOS-14 Conference (2024 August). Andrew has extensive experience in severe accident phenomena and performing integrated analysis for a variety of reactor systems (BWR, PWR/iPWR, CANDU, HTGR, etc). Andrew holds a PhD in Engineering Physics from McMaster University as well as masters (Engineering Physics) and bachelor (Engineering Physics and Management) degrees from McMaster. Andrew is the Chair of the Chalk River Branch of the Canadian Nuclear Society and is an EIT with the Professional Engineers of Ontario (PEO).

Dr. Nicolas TRÉGOURÈS works in the field of nuclear safety, with a particular emphasis on how nuclear fuel behaves under different conditions. His expertise covers nuclear fuel behavior during normal operation, incidental events like LOCA and RIA, spent fuel pool storage, and accident-tolerant fuels (ATF).

He has held several positions at the French Institut de Radioprotection et de Sûreté Nucléaire (IRSN, now ASNR). He is currently Department Deputy Head where he coordinates, manages, and monitors R&D activities. He has served as the Scientific and Technical Manager of the ANR IRSN DENOPI project dedicated to the experimental study of spent fuel pool under loss of cooling and loss of coolant accident conditions.

He is actively involved in OECD/NEA activities, currently serving as the Vice-chairman of the

Working Group on Fuel Safety (WGFS), Chairman of the "Review of key Halden Legacy Data Sets" task and Chairman of the OECD International Standard Problem #53. Furthermore, he is the organizer and animator of the Analytical Group of the OECD CABRI International Project (CIP).
Dr Nicolas Trégourès holds a Ph.D. degree in Physics from Université Joseph Fourier (Grenoble, France) in 2001.

Dr. Kostadin IVANOV has been working in the Nuclear field for over 34 years. His field of expertise includes: Reactor Physics; Methods in Static and Dynamic Analysis; Nuclear Power Plant Modeling and Safety; Fuel Management and Core Design; Verification and Validation of Multi-Physics simulations. He has worked over ten years for the Pennsylvania State University as Distinguished Professor including Graduate Coordinator of Nuclear Engineering Programme, and for two years as Visiting Professor at the Karlsruhe Institute of Technology, Department of Mechanical Engineering, in Germany. Since 2015 he is Professor and Department Head and at present Distinguished Professor at the Department of Nuclear Engineering at the North Carolina State University (USA). He was and currently is in charge as leading scientist of several international benchmarks organized under the umbrella of the Nuclear Energy Agency (NEA). His work has been published on hundreds of academic and international journals and conference proceedings. He holds a PhD in Reactor Physics from the Bulgarian Academy of Science.

Dr. Bruno TOURNIAIRE, EDF senior Expert in severe accident and WGAMA technical advisor, has been working in the nuclear safety and severe accident fields for around 25 years. After a PhD at the Ecole Centrale de Lyon, he joined the French Institut de Protection et de Sûreté Nucléaire (now ASNR) to work on fire safety. He joined the Commissariat à l'Energie Atomique (CEA) in 2000 to work on severe accident and then Electricité de France (EDF) in 2009 where he became senior Expert in 2021. His field of expertise covers the main issues related to severe accidents such as molten core concrete interaction, source term, hydrogen risk, in-vessel retention, etc. He has several positions and activities at EDF including coordination and management of the R&D on severe accident, safety studies for the French NPP, development of models and methodologies for severe accident analysis. He also contributes to the knowledge management at EDF. Dr B. Tourniaire is involved in several R&D OECD programs as EDF representative both in Program Review Group and Management Board (ROSAU, ESTER, QUENCH-ATF).

Dr Luca PODOFILLINI is a senior scientist at the Paul Scherrer Institute of Switzerland. He has a Nuclear Engineering degree and Ph.D. in Nuclear Engineering from the Polytechnic of Milan (2004). His current activities include Human Reliability Analysis research and regulatory support tasks for the Swiss Federal Nuclear Safety Inspectorate. His research addresses the development of quantitative models of human performance in industrial systems, with focus on errors in decision-making, dynamic safety assessment, collection of data in simulated emergencies. He is co-author of about 40 publications on peer-reviewed international scientific journals.

Dr. Sanjeev GUPTA obtained his Ph.D. in Fluid Mechanics from IMT Atlantique in Nantes, France. Since 2008, he has held leadership roles in experimental programs related to nuclear safety. Prior to that, he was involved in large-scale experimental test facilities in the field of nuclear safety research across prominent European research institutions. The current focus of his work is on thermal-hydraulics and severe accident research in water-cooled large water reactors and SMRs.

He served as program manager for the international joint nuclear safety project NEA /THEMIS and is actively involved in NEA documentation efforts and task forces, particularly as a member of the Working Group on Analysis and Management of Accidents (WGAMA). Dr. Gupta also coordinated the international in-kind project IPRESCA, focused on source term research.

He is a member of the Advisory Boards for ongoing European Commission projects including AMHYCO (combustible gas risk management) and SASPAM-SA (related to Small Modular Reactors). Dr. Gupta has been actively involved in European nuclear activities, such as SNETP (Sustainable Nuclear Energy Technology Platform), ETHC (European Thermal Hydraulics Community), and the European SMR Industry Alliance, and he contributes to

IAEA activities as an expert and lecturer.

Mr. Klaus UMMINGER worked as scientific senior expert and project manager in the thermal-hydraulic research center of Framatome (former Siemens/KWU and AREVA) in Erlangen, Germany. After graduating from the Technical University of Munich (Mechanical Engineering), he joined Framatome in 1982, where he was engaged in experimental projects in the field of reactor safety research and in the design of fluid instrumentation. Since 1995 Klaus Umminger had been in charge of the large experimental program PKL. The PKL facility has been used to investigate the T/H system behavior of PWRs during postulated accidents. Since 2001 the PKL project has been continued as an international OECD project. He was head of the PKL working group with participation of experts from utilities, licensing authorities and industry and participated in other international programs on reactor safety research. Main areas of work: Design/performance/interpretation of experiments for application in code validation and experimental verification of AM measures for PWRs.

After retiring from Framatome, Klaus Umminger is now chairman of the SILENCE international network of Organizations involved in the development and exploitation of "significant" thermal-hydraulic experiments.

Dr. Martina ADORNI, WGAMA Secretariat, serves as a Nuclear Safety Specialist at the Nuclear Energy Agency (NEA) within the Division of Nuclear Safety Technology and Regulation. She provides expert technical support to working groups focused on accident analysis, management, and safety research as well as to nuclear safety joint projects.

Dr Adorni is a Nuclear Engineer, holding a PhD in Nuclear and Industrial Safety obtained at the University of Pisa (Italy). Her research focused on accident analysis in research reactors and the development of a methodology for the analysis of fuel behavior during accident conditions in nuclear power plants. Her expertise spans severe accidents, fuel behavior, thermal-hydraulics, and safety analysis using computer codes.

Throughout her career, Dr. Adorni has contributed to numerous national and international projects, including coordinating the analytical activities of the NEA Sandia Fuel Project

Phase I. Following her PhD, she worked at Bel V (Belgian TSO) as an R&D Coordinator and Safety Analyst, where she analysed severe accident progression and source term assessment using computer codes. She participated to the evaluation of the nuclear safety assessments related to severe accidents and PSA such as filtered containment venting, steam explosion, mitigation strategies, spent fuel pool, design extension conditions. She led the ETSON Research group since 2018.